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Validation of Monte Carlo Neutron Physics Codes for Fully Ceramic Microencapsulated PWR Fuel Lattice

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Outline

- A brief on TRISO particle & FCM fuel
- History of TRISO fuel
- Concept of PWR with FCM
- Treatment of double heterogeneity
- Monte Carlo Codes (MVP & Serpent)
- Pin Cell Validation
- Lattice Level Validation
- Summary/Conclusion



A brief on TRISO and FCM fuel

- TRISO Particles
 - Tiny spherical particle containing fuel (kernel).
 - Layers surrounding kernel (Buffer, IPyC, SiC, OPyC).
- FCM Fuel

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- TRISO Particles highly packed in SiC Matrix



History of TRISO fuel

- Invention and Initiation of TRISO Manufacturing
 - 1957: Invented by R. Huddle in Harwell, UK
 - 1964-75: DRAGON, UK .1st demonstration HTGR
 - 1966-1988: AVR Project, Jülich, Germany. Pebble-bed
 - 1967-1974: Peach Bottom-1, Pennsylvania, USA. HTGR
 - 1998 : HTTR project, Japan. HTGR
 - 2000: HTR-10, China. Pebble-bed
 - 2011: FCM Concept, FCRD Deep Burn Program, US-DOE



Concept of PWR with FCM Fuel

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Treatment of Double Heterogeneity

Double Heterogeneous Fuel

Level-1: Multi Layered Nature of TRISO Level-2: Fuel Rod/Assembly Geometry

Direct Treatment

Implicit/Explicit Modeling in MC Code

In-Direct Treatment

Homogenization of Fuel Region



Monte Carlo Codes (MVP & Serpent)

- Monte Carlo codes
 - MVP by JAEA, Japan using JENDL-3.3
 - Serpent by VTT, Finland using ENDF/B-VII.0
 - Capable of modeling doubly heterogeneous geometries like Pebble bed, HTGR or FCM in implicit and explicit manner.

Can perform burnup calculations for DH geometries.

- High computational resources to reduce the statistical fluctuations to less than 4 pcm for eigenvalues.
 - For solid fuel case, 100,000 particles 1100 batches
 - FCM Case, 100,000 particles in 5500 batches



Pin Cell Validation

- VERA benchmark revision 4. with KENO-IV using ENDF/B-VI.1
 - Pin cell validation
 - K-infinity estimation against various conditions.
 - *Transformation of solid UO*₂ *pin cell into FCM pin cell.*

FCM pin cell validation							
Problem #	Temperatur	e (°K)	Moderator Density				
	Moderator	Fuel	(g/cm ³)				
1A	565	565	0.743				
1 B	600	600	0.661				
1C	600	900	0.661				
1D	600	1200	0.661				



Description	Value/Unit
Fuel Radius	0.4095 cm
He-gap thickness	0.0085 cm
Clad thickness	0.047 cm
Pin pitch	1.26 cm
UO ₂ Enrichment	3.11 w/o
Boron	1300 ppm

Case #	Reference	Unc. (±)	Serpent	Unc. (±)	R/Err	MVP	Unc. (±)	R/Err
1A	1.187038	0.000054	1.18572	0.000046	-0.11%	1.18372	0.000041	-0.28%
1 B	1.182149	0.000068	1.18131	0.000048	-0.07%	1.18037	0.000046	-0.15%
1C	1.171720	0.000072	1.17121	0.000049	-0.04%	1.16984	0.000045	-0.16%
1D	1.162603	0.000071	1.16266	0.000050	0.00%	1.16098	0.000046	-0.14%

- Max relative error of Serpent is 0.11 %
- Max Relative error for MVP 0.28 %.
- Excellent agreement



Pin cell Validation contd.

- Transformation of standard fuel pin into FCM fuel pin
- Zircaloy, FeCrAl Clad
- PF=0.50
- Transformation:

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• Fuel: UO_2 Solid $\rightarrow UC$ FCM Enr. : 3.11 w/o $\rightarrow 12.65$ w/o



Case#	Serpent	Unc. (±)	MVP	Unc. (±)	R/Err
1A	1.398810	0.000037	1.405150	0.000035	0.45%
1 B	1.411880	0.000037	1.415570	0.000035	0.26%
1C	1.401970	0.000038	1.405090	0.000037	0.22%
1D	1.393530	0.000039	1.395840	0.000038	0.17%
2					



Pin cell Validation contd.

• FeCrAl Clad case

Problem #	Serpent	Unc. (±)	MVP	Unc. (±)	Rel Diff
1A	1.387350	0.000038	1.394180	0.000036	0.49%
1B	1.400630	0.000038	1.405520	0.000037	0.35%
1C	1.391020	0.000039	1.395360	0.000037	0.31%
1D	1.382550	0.000039	1.386530	0.000038	0.29%

- Max relative difference of MVP is
 - ✓ 0.45% for Zircaloy Clad Case
 - ✓ 0.49% for FeCrAl Clad Case
- Excellent agreement found in both cases



Lattice Level Validation

- Lattice level validation, Problem#2C
 - K-inf values against HFP
 - Pin power distribution (1/8 FA)
 - *Transformation of solid UO*₂ *lattice into FCM lattice.*
 - Validation of FCM results
 - Fuel Assembly Pitch = 21.5 cm
 - Inter Assembly half gap = 0.04 cm
- Guide Tube inner radius = 0.561 cm
- Guide Tube thickness = 0.41 cm



1.26 cm

- Hot Full Power (HFP) Condition
 - Moderator Temp = $600 \text{ }^{\circ}\text{K}$
 - Fuel Temperature = $900 \text{ }^{\circ}\text{K}$
 - Moderator Density = 0.661 g/cm^3



• K-infinity

Case#	Reference	Unc. (±)	Serpent	Unc. (±)	R/Err	MVP	Unc. (±)	R/Err
2C	1.173751	0.000023	1.17339	0.000021	-0.03%	1.17169	0.000019	-0.18%





• Relative Pin Power Distribution *contd*.



3b) Pin-by-pin Power Distribution by SERPENT

0.21	0.30							
0.22	0.26	0.17						
	0.19	0.22						
0.01	0.17	0.20	0.24	0.18				
0.21	0.21	0.19	0.12	0.15				
	0.10	-0.27		0.09	0.04	0.01		
0.03	0.03	0.00	-0.05	-0.03	-0.12	-0.09	-0.31	
-0.22	-0.22	-0.27	-0.25	-0.21	-0.29	-0.39	-0.55	-0.73

3c) SERPENT Relative Error (%)



3d) Pin-by-pin Power Distribution by MVP

0.28	0.28							
0.23	0.49	0.22						
	0.48	0.52						
0.04	0.47	0.45	0.53	0.23				
0.21	0.40	0.40	0.38	0.43				
	0.29	-0.01		0.34	0.14	-0.30		
-0.13	0.04	0.04	0.06	-0.07	-0.22	-0.28	-0.90	
-0.83	-0.57	-0.58	-0.56	-0.62	-0.75	-1.00	-1.26	-2.10

MVP Relative Error (%)

• FCM Assembly



- Conservation of initial fissile content
- Packing fraction = 0.50

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- $UO_2 \longrightarrow UC$; Enrichment: 3.11 w/o \longrightarrow 12.65 w/o
- All other dimension, material specs and conditions are same

工程计算物理实验室

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• K-infinity and Relative Pin Power Distribution



- Excellent agreement of K-infinity values for pin cell and lattice cases for both reference and FCM.
- Relative pin power distribution for both cases is in very good agreement
- Both codes predicted the exact location for hottest pin

Thus use of Serpent and MVP for neutronic studies of FCM loaded PWR will provide reliable results





